

DCISC

DIABLO CANYON INDEPENDENT SAFETY COMMITTEE

COMMITTEE MEMBERS

NAJMEDIN MESHKATI
PER F. PETERSON
RALUCA O. SCARLAT

WEBSITE - WWW.DCISC.ORG

By Email Only to kara.woodruff@sen.ca.gov
Original will **not** follow.

August 17, 2026

Kara Woodruff, JD/MA
District Director & Senior Policy Advisor
Senator John Laird (SD-17)

Dear Ms. Woodruff:

This will respond to your email to DCISC Legal Counsel Robert Rathie on July 15, 2026, where you posed certain questions on behalf of Senator Laird regarding matters in connection with the recent testing performed by Westinghouse Electric Company of surveillance Capsule B from the Unit 1 Reactor Pressure Vessel at Pacific Gas & Electric Company's Diablo Canyon Nuclear Power Plant in Avila Beach, California.

The Westinghouse Capsule B report (ML26180A296) was submitted on June 29, 2026 to the NRC to demonstrate PG&E's regulatory compliance with Appendix H to 10 CFR Part 50. This report together with its PG&E transmittal letter to the NRC references the specific regulations and standards that govern the Reactor Vessel Surveillance program at Diablo Canyon (e.g. 10 CFR 50, 10 CFR 54). Referencing studies other than what is required and accepted by the current NRC regulation *would not have been necessary* as part of reporting surveillance capsule results to the NRC. For this reason, PG&E and Westinghouse did not discuss the possible future evolution of NRC regulations in the Capsule B report.

Regulatory evolution on the topic of embrittlement was discussed by DCISC at the June 2026 Public Meeting; see June 17th, 2026 recording time-stamp 4:02 to 4:35, accessible at: https://slo-span.org/meeting/dcisc_20260616/. A 2023 ASTM paper by the NRC authors Widrevitz and Poehler was among the topics of conversation:

D. S. Widrevitz and J. C. Poehler, "History and Development of U.S. NRC Regulatory Guide 1.99," in "Radiation Embrittlement Trend Curves and Equations and Their Use for RPV Integrity Evaluations," ed. W. L. Server, M. Brumovský, and M. Kirk (West Conshohocken, PA: ASTM International, 2023), 398–416, <http://doi.org/10.1520/STP164720220041>. ASTM Symposium on Radiation Embrittlement Trend Curves/Equations and Their Use for RPV Integrity Evaluations held in Prague on April 19–22, 2022.

The ASTM 2023 paper by Widrevitz and Poehler describes the history and development of Regulatory Guide 1.99, and raises concerns about the use of NRC's correlations for neutron embrittlement for neutron doses above 3×10^{19} n/cm². Neutron doses exceeding 3×10^{19} n/cm² would only be reached if DCP Unit 1 operated for more than 80 years.

The relevance of the NRC rulemaking effort and the corresponding ASTM 2023 paper to aging management of Diablo Canyon Unit 1 is also described in the memorandum titled "Memo: Reactor Vessel Embrittlement Regulations," by DCISC technical consultant Dr. Mark Kirk, dated June 9th, 2026, and accessible on the DCISC website at: <https://www.dcisc.org/download/library/topical-library/memo-reactor-vessel-embrittlement-regulations-106.pdf>.

This ASTM 2023 paper is part of on-going communication from the NRC concerning its effort to update its embrittlement regulationsⁱ. NRC staff requested authorization from the NRC Commissioners to begin rulemaking on this topic in 2022 (SRM-SECY-22-0019). The staff was authorized by the Commission to begin that work in 2025. In June 2026 the NRC issued a "*white paper [that] discusses NRC staff's preliminary technical perspectives developed in response to the direction in SRM-SECY-22-0019 [...] intended solely to describe the staff's initial thoughts and to support stakeholder engagement.*" Thus, as of the date of this letter, the NRC has issued neither revised guidance nor revised rules.

For aging management of the DCP reactor pressure vesselsⁱⁱ, there are several tools at the disposal of reactor engineers (a) to inspect, and (b) to manage the aging of the reactor pressure vessel. For example, if exceeded, the NRC's PTSⁱⁱ screening criteria prompts additional reactor-specific analysis, or mitigation measures. The adjusted reference temperature (ART) also informs the Pressure - Temperature Limits for Unit 1 and Unit 2. Therefore, an important goal of reviewing the Capsule B results is deciding *what the next set of analysis or mitigation strategies ought to be, if any*.

ⁱ As an example of the type of regulatory developments that the paper proposes, it indicated that NRC Regulatory Guide 1.99 will begin to under-predict the transition temperature at total fast neutron dose in excess of 3×10^{19} n/cm² (see Table 1 for the relevance of this fluence to DCISC Unit 1), especially for the base metal, and more so beyond 6×10^{19} n/cm². The capsule B data points (plant-specific data) fall in the early region of what the NRC staff have described as higher scatter. However, the inner wall of the reactor vessel in Unit 1 cannot reach 3×10^{19} n/cm² until the year 2070 (see Table 1), which is 25 years after expiration of the plant license.

ⁱⁱ Neutron irradiation causes aging (in the form of embrittlement) of the reactor vessel. The magnitude of irradiation dose is quantified by fluence of fast neutrons that the material has experienced at a particular plane in the material (n/cm²). As for all ferritic steels, the reactor pressure vessel steel experiences a transition from brittle behavior at low temperatures to ductile behavior at high temperatures. The temperature at which the steel transitions from brittle to ductile behavior increases with increased irradiation dose. The "limiting material" in a reactor pressure vessel is the steel that has a transition temperature closest to the NRC's pressurized thermal shock screening (PTS) criteria found in 10 CFR 50.61. The limiting material for Unit 1 is the longitudinal weld 3-442C, which was last ultrasonically-inspected following the requirements of the ASME Code in 2025. The thickness of the reactor pressure vessel is more than 8 inches at the beltline, and the inner wall of this weld will, by the end of 2026 have experienced approximately 1.5×10^{19} n/cm² fast neutron fluence (this fluence represents approximately 36¼ effective full power years (EFPY) of operation at Unit 1). Neutron irradiation scales with power level and hence energy produced; since reactors sometimes operate at reduced power and have outage periods for refueling and maintenance, the EFPY is less than the age of the reactor. For Unit 1, the age of the reactor is referred to as the years that have passed since 1985, irrespective of EFPY of operation. EFPY, and hence fluence, will depend on the power level and duration of outages of the specific unit.

You have also asked about future capsule testing at intervals of two or five years. The ASTM 2023 paper discusses “*additional surveillance testing requirements for long-term operation [...] (e.g., beyond 60 years of operation),*” in light of larger data scatter observed at fluences above $3 - 6 \times 10^{19}$ n/cm². Subsequently, the 2026 NRC White Paper proposes a revision of surveillance requirements “*to ensure that data [...] with a neutron fluence equal or greater than the peak RPV neutron fluence for extended operation is tested at an appropriate periodicity (e.g., every 20 years);*” and it recognizes that “*some licensees may have already obtained surveillance data that supports long term operation of the RPV.*”

Section 3.1.1. Surveillance. in “Potential Changes to Embrittlement And Surveillance Requirements For High-Fluence Nuclear Power Plants In Long-Term Operation under SRM-SECY-22-0019,” Nuclear Regulatory Commission, [ML26148A433](#), June 2, 2026.

The samples of the limiting material removed with Capsule B had accumulated a fast neutron fluence of 3.91×10^{19} n/cm², which would be reached after approximately 105 years of operation. The current scope of DCISC does not include considerations of operations beyond 60 years, or 2.14×10^{19} n/cm², which corresponds to fluence on the inner surface of the Unit 1 vessel (See Table 1). Since currently available surveillance capsule data for Unit 1 exceeds this value, there would not be a benefit to testing of coupons at even *higher* fluence values.

Table 1. Timeline summary for DCP Unit 1 that correlates the values of fluence with the corresponding operational calendar year, age of reactor, and effective full-power years (EFPY) when these fluence levels have been or could be reached for the Unit 1 RPVⁱⁱⁱ. Values are derived from Tables 6-6, 7-1, and 5-10 in Capsule B report (ML26180A296); after 2045, hypothetical projections are made assuming 1 EFPY per year and $(0.04)10^{19}$ n/cm² per EFPY.

Fluence (10^{19} n/cm ²)	Approximate Year Reactor Reaches this Fluence	Corresp. Reactor Age (years)	Corresp. EFPY	Irradiated samples with corresponding fluence
0.293	1993	8	5	Capsule S
1.07	2015	30	26	Capsule Y
1.24	2020	35	31	Capsules C&D (removed from reactor, not analyzed)
1.35	2024	39	34	Capsule V
1.42	2026	41	36	
1.6	2030	45	41	(fluence and EFPY estimated by interpolation)
2.14	2045	60	54	(values per Amendment 1 to the PG&E license renewal application, ML24289A118)
3	2066	81	75	(hypothetical projection)
3.62	2083	98	92	Capsule B (surrogate weld)
3.91	2090	105	99	Capsule B
6	2143	158	152	(hypothetical projection)

ⁱⁱⁱ Fluence represents fast neutron fluence on the inner face of the reactor pressure vessel (RPV); the maximum of across all axial and azimuthal positions on the RPV is reported.

Letter to Kara Woodruff
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Finally, you have asked about the experience of the authors and reviewers of the Westinghouse report. This question falls within the current scope of review by DCISC members. An extensive update of Capsule B will be provided at the upcoming October 2026 Public Meeting, and some background information is provided here. Westinghouse has corporate experience in the performance and reporting of surveillance capsule results dating back to the 1970s. Diablo Canyon is a Pressurized Water Reactor, and Westinghouse performs the testing on virtually all capsules from pressurized water reactors in the USA, of which there are 60 operating units designed and supplied by Westinghouse or Combustion Engineering (acquired by Westinghouse). Quality assurance programs certify the quality of the data and the transparent documentation of the quality and include such considerations as training, expertise, review and audits. It falls within the current DCISC scope to review the metrology and corresponding quality assurance and quality control pertaining to data reported in the Capsule B report. DCISC has had meetings and correspondence with Westinghouse on this topic and will continue this inquiry.

If you or the Senator have further questions, please feel free to contact me. Perhaps, and if you concur, we could arrange a mutually convenient date and time prior to the October 20-21, 2026 DCISC public meeting for a Zoom remote meeting with you and the Senator (and perhaps including San Luis Obispo Supervisor Gibson) to pose questions and discuss with a Committee Member together with technical consultants Dr. Kirk and Dr. Andrew C. Kadak who have for some time now been involved in the assessment of the integrity of the Unit 1 reactor pressure vessel with reference to embrittlement.

Sincerely,



Najmedin Meshkati

Chair, Diablo Canyon Independent Safety Committee

MK:rwr

cc: DCISC Members
DCISC Technical Consultants

From: Woodruff, Kara <Kara.Woodruff@sen.ca.gov>
Sent: Wednesday, July 15, 2026 1:08 PM
To: Info@DCISC.org
Subject: Re: To IPRP re DCPD Unit 1 Capsule B Technical Report & Letter to DCISC from Senator Laird
Attachments: dcisc History and Development of U.S. NRC Reg Guide 1.99.pdf

Hi Bob -- thanks for your time today. I wanted to mention some issues that were raised by local reviewer/expert of the Westinghouse Capsule B report, which may be of interest to members of the DCISC and Dr. Kirk.

First, the Westinghouse report appears to not reference the (attached) article by Widrevits and Poehler, which suggests that the model used to assess reactor embrittlement may be insufficient beyond 60 years of reactor operation. Why was this article not referenced in the Westinghouse report? Is it inapplicable, or unpersuasive? Given that the Westinghouse report concludes that the testing results "confirm" that Diablo Canyon's Unit 1 continues to be safe for "at least" 60 years of operation it seems appropriate that this article should have been mentioned/addressed.

Second, and related, if the Widrevits and Poehler article raises questions about embrittlement testing models as reactors approach 60 years of operations, what does that mean for future testing of both Unit 1 and Unit 2 of Diablo Canyon during the remaining life of the plant? Would an embrittlement testing requirement of, say, every five years for both units, with more frequent testing (every two years?) be appropriate if/as plant operations approach year 2045?

Finally, we don't know a lot about the authors/reviewers of the Westinghouse report. Do they have Ph.D.'s in applicable fields? What is their level of expertise and experience, and are those adequate and acceptable to the members of the DCISC and Dr. Kirk in supporting the conclusions of the report?

Thank you for your time and attention, Kara

Kara Woodruff, JD/MA
District Director & Senior Policy Advisor
Senator John Laird (SD-17)
Cell: 805/440-6650
CA Capitol Office: 916/651-4017
Email: kara.woodruff@sen.ca.gov
<https://sd17.senate.ca.gov/>

From: Woodruff, Kara
Sent: Monday, July 13, 2026 3:42 PM
To: Info@DCISC.org; Dawson, Timothy@DOC; Seitz, Gordon@DOC; Johnson, Philip@Coastal; Bruce Gibson

Cc: Wardrip, Jonathan

Subject: Re: To IPRP re DCP Unit 1 Capsule B Technical Report & Letter to DCISC from Senator Laird

Thank you, Bob and everyone. Senator Laird wrote to the DCISC to ask that the committee ensure that an independent, third-party entity with appropriate credentials provide peer review of the Westinghouse embrittlement testing results for Unit 1 of Diablo Canyon Power Plant that were recently made available to the public.

As you note, the DCISC will take up this issue in October and while that timing is further off than desirable, it seems appropriate for the committee to establish a subcommittee comprised of experts in the field of material science/nuclear energy, or otherwise as would be needed to adequately review the Westinghouse analysis and results. This is consistent with what the DCISC did in reviewing questions about the seismic condition at/surrounding the plant, when it set up the Seismic Review Team.

If/when such a peer review process proceeds, it may be of use for such a team to review the attached reports, which provide some background on embrittlement testing and the models under which that is conducted. I do not believe they were all referenced in the Westinghouse analysis.

Thank you for your consideration, Kara

Kara Woodruff, JD/MA
District Director & Senior Policy Advisor
Senator John Laird (SD-17)
Cell: 805/440-6650
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Email: kara.woodruff@sen.ca.gov
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From: Info@DCISC.org <info@dcisc.org>

Sent: Saturday, July 11, 2026 12:09 AM

To: Dawson, Timothy@DOC; Seitz, Gordon@DOC; Johnson, Philip@Coastal; Bruce Gibson

Cc: Info@DCISC.org; Woodruff, Kara; Wardrip, Jonathan

Subject: To IPRP re DCP Unit 1 Capsule B Technical Report & Letter to DCISC from Senator Laird

Dear IPRP Representatives-

In accordance with the direction given me at the DCISC's June 17 public meeting, I am provided you as attachments Senator John Laird's letter to the DCISC on June 15 in which he states his position that the IPRP should review and concur in the results of the Unit 1 RPV Capsule B Technical Report.

I have copied Kara Woodruff, the Senator's District Director, in case she wants to comment.

The DCISC received the report on June 29 and briefings have been conducted by PG&E and Westinghouse with all three DCISC members, to be documented in fact finding reports. The DCISC expects to address this matter on the agenda at its October 20-21, 2026, public meeting in Avila Beach.

Please let me know of any questions and thank you for your courtesy and the continuing cooperation between the DCISC and the IPRP.

Bob Rathie
DCISC Legal Counsel
info@dcisc.org

From: Info@DCISC.org
Sent: Wednesday, July 1, 2026 11:09 PM
To: Bruce Gibson; Info@DCISC.org
Cc: Woodruff, Kara; Jim Dantona; Info@DCISC.org
Subject: RE: DCP/ Westinghouse Capsule B report

Bruce –

Thank you for your email yesterday and apologies for my late reply (it's been a busy couple of days). This will acknowledge receipt of and thank you for this communication which I provided yesterday to our Members and the Technical Consultants.

The Capsule B report is now on our website homepage at www.dcisc.org along with the PowerPoint presentation used at the briefing yesterday provided by DCP/ Westinghouse to Dr. Meshkati and CEC Chair Hochschild and members of his staff.

Each Committee member will receive an informational briefing from DCP/ Westinghouse and have an opportunity to pose questions and it is our intent to have Drs. Kirk and Kadak review the report initially and to make their review available on our website prior to the October 20-21 public meeting and to schedule discussion at that meeting. In the interim the Committee Members are actively reviewing and considering what next steps they intend to take - including concerning your and Senator Laird's request for peer review.

Cordially,
Bob

From: Bruce Gibson <bgibson@co.slo.ca.us>
Sent: Tuesday, June 30, 2026 9:49 AM
To: Info@DCISC.org
Cc: Woodruff, Kara <kara.woodruff@sen.ca.gov>; Jim Dantona <jdantona@co.slo.ca.us>
Subject: DCP/ Westinghouse Capsule B report

Hi Bob — You may have seen PGE's public announcement of the results of the Capsule B testing. As part of those communications, they are apparently making the report public – at east Sen. Laird's office and I have received copies, with no stated restrictions regarding further release.

I'd be interested to know how this affects the DCISC's consideration of the report — I was under the impression that the Committee was to be briefed in July and I'm wondering whether DCISC is planning to discuss these results at one of your upcoming public meetings.

I'm also writing to request that DCISC have the report peer reviewed before rendering your final conclusions. Both Sen. Laird and I are strongly of the opinion that peer review would be an important assurance to the public regarding the safety implications of the report's findings. As you know, peer review of seismic hazard investigations have been an important part of assuring transparent consideration of those very technical data and this assessment of potential embrittlement has similarly complex technical issues at play. I should note that I'm making this request as a SLO County elected official and not as a member of the Independent Peer Review Panel, which works on seismic hazard.

Thanks very much for your consideration and I look forward to catching up with you and the Committee soon.

Sincerely,

Bruce Gibson
Supervisor, District 2
San Luis Obispo County
Office: 805-781-4338

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California State Senate

**SENATOR
JOHN LAIRD**

SEVENTEENTH SENATE DISTRICT



STANDING COMMITTEES
BUDGET & FISCAL REVIEW
CHAIR
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LABOR, PUBLIC EMPLOYMENT
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NATURAL RESOURCES & WATER
RULES
JOINT COMMITTEES
LEGISLATIVE BUDGET
CHAIR
RULES
VICE CHAIR
SELECT COMMITTEE
NONPROFIT SECTOR

June 15, 2026

Diablo Canyon Independent Safety Committee
Office of the Legal Counsel
SW 4th & Mission, Suite 2, PO Box 4523
Carmel, CA 93921-4523

Subject: DCISC Agenda for June 17, 2026
Item No. XXXII 6. - Unit 1, Capsule B Embrittlement Testing

Dear Members of the Diablo Canyon Independent Safety Committee:

As you know, I'm the State Senator representing the Central Coast including the Diablo Canyon Power Plant and surrounding lands, as well as most of San Luis Obispo County. I'm writing about the subject item, concerning embrittlement testing of Unit 1's Capsule B.

The integrity of Unit 1 fits into a larger discussion about the possible extension of Diablo Canyon beyond 2030. For that reason, and by way of background, I am attaching a letter recently sent to the San Luis Obispo Board of Supervisors.

As mentioned in the attached letter, it is imperative that if Diablo Canyon is extended beyond 2030, we must first confirm that the plant will continue to operate safely. One of the key outstanding safety questions is whether the embrittlement test will prove that indeed Unit 1 is safe for years to come.

When Capsule B was removed last year, PG&E stated that the results of that test would be broadly available by April 2026. However, two months later, no results are forthcoming. We understand now that the results may not be available until October 2026. It is my position that Diablo Canyon Power Plant should not be extended until after those results are available and, further, that they definitively demonstrate the integrity of Unit 1.

Given the circumstances surrounding the extraction of Capsule B, it is further my position that when the results are available, they be subject to the review and concurrence of not only the DCISC, but also the Diablo Canyon Independent Peer Review Panel. This independent review is not only consistent with standard science protocols, it will provide greater confidence by the community that Diablo Canyon Power Plant is safe to operate, for years to come.

Thank you for your consideration.

Sincerely,

A handwritten signature in black ink, appearing to read "John Laird". The signature is fluid and cursive, with the first name "John" and last name "Laird" clearly distinguishable.

JOHN LAIRD
Senator, 17th District

Info@DCISC.org

From: Info@DCISC.org
Sent: Wednesday, June 10, 2026 5:26 PM
To: 'Bruce Gibson'; Info@DCISC.org
Cc: Jim Dantona; Woodruff, Kara
Subject: RE: DCISC discussion of Capsule B testing requirements - June 17

Dear Supervisor Gibson – thank you for your email and for taking your time to speak with me yesterday.

I will of course provide your email with the request for peer review of the final Westinghouse Capsule B report to our Members and Technical Consultants.

I look forward to seeing you for at least part of the DCISC meeting next week in Avila Beach, and I thank you for your service to the County, the IPRP and interest in the DCISC and its activities.

Cordially,
Bob Rathie
(831) 578-5043

From: Bruce Gibson <bgibson@co.slo.ca.us>
Sent: Wednesday, June 10, 2026 2:52 PM
To: Info@DCISC.org
Cc: Jim Dantona <jdantona@co.slo.ca.us>; Woodruff, Kara <kara.woodruff@sen.ca.gov>
Subject: DCISC discussion of Capsule B testing requirements - June 17

Dear Mr. Rathie — I notice that the DCISC will hold a discussion of the regulatory requirements and technical standards for the testing and analysis of Reactor Surveillance Capsule B at your upcoming meeting on June 17. I write to request that the DCISC require peer review of that report when it is available.

I understand that the DCISC has not yet received a copy of the Westinghouse Capsule B report, as had originally been planned for April or May. Further I understand that the next action on that matter is to be a confidential briefing to your committee in June or July.

Given 1) the significant delay in receiving this information, 2) the pending deadline of submission to the NRC in October, and 3) the need for public transparency on this important matter, I strongly urge the DCISC to require a qualified peer review of the final Westinghouse Capsule B report as part of your public consideration of this matter.

As a member of the CPUC's Independent Peer Review Panel (IPRP) on seismic hazard, I have seen firsthand the importance of independent peer review in maintaining public trust in the oversight responsibilities with which the DCISC and IPRP are charged.

Thank you for your consideration and please don't hesitate to contact me if you have any questions.

Sincerely,

Bruce Gibson
Supervisor, District 2
San Luis Obispo County
Office: 805-781-4338

DCISC

DIABLO CANYON INDEPENDENT SAFETY COMMITTEE

COMMITTEE MEMBERS

WEBSITE - WWW.DCISC.ORG

NAJMEDIN MESHKATI
PER F. PETERSON
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Copy Via Email to:

bxjk@pge.com

May 5, 2026

Mr. Blair Jones
Vice President, Business & Technical Services
Pacific Gas & Electric Company
Diablo Canyon Power Plant
Avila Beach, California

Re: **Recent Meeting with the Chair of the California Energy Commission.**

Dear Mr. Jones:

On April 30, 2026, DCISC Member Dr. Meshkati and Legal Counsel Mr. Rathie met in Sacramento with the Chair of the California Energy Commission Mr. David Hochschild and members of his senior staff together with staff from the office of CEC Vice Chair Mr. Siva Gunda. The purpose of the meeting was to review the Committee's current activities and topical areas of inquiry and to answer questions and respond to requests.

Among these topics was the status and plans for the DCISC's review of evaluations of DCP Unit 1 surveillance Capsule B. During the discussion the DCISC representatives explained that, while PG&E sought to expedite the process, a period of 12-months is usually the minimum to complete the necessary analyses to assess a capsule's embrittlement properties. We understand that since the capsule was removed from the vessel in April 2025 there have been delays due to equipment issues experienced by the Westinghouse testing facility and the report is not expected to be received, reviewed by PG&E and made available to the NRC until sometime in the Q3 2026.

The DCISC representatives also explained that, while previously the Committee expected to receive an advance copy of Westinghouse's report, designated as confidential, prior to its submission to the NRC to allow for an expedited review by the Committee and its Consultant Dr. Mark Kirk, PG&E currently plans to provide the report to the Committee at the same time that it is submitted to the NRC and made available to the public.

Letter to Mr. Blair Jones
May 5, 2026
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Mr. Hochschild requested that the DCISC make a request that PG&E provide a written response to the Committee explaining in more detail why more than one year is being required to complete the embrittlement evaluation process. He specifically asked why the report is late and suggested that the Westinghouse report be provided to the Committee at the earliest practicable opportunity, in advance and prior to its submission to the NRC, as also discussed at the DCISC's February 2026 Public Meeting. Mr. Hochschild further emphasized that the embrittlement issues is one of his top safety priorities and urged the Committee to request that PG&E reconsider its position and provide an advance copy of the Westinghouse report to the Committee. Hence this letter.

Thank you for your attention to this important matter and I hope to hear from you within ten business days. Should you have any questions both Dr. Meshkati or Mr. Rathie are available to assist.

Sincerely,



Per F. Peterson
DCISC Chair

PFP:rwrr

cc/email: CEC Chair Mr. David Hochschild

Dr. Justin Cochran, Emergency Coordinator/Nuclear Advisor to CEC Chair

Ms. Carina Corral, Strategic Initiatives Manager, DCPD

DCISC Members

DCISC Technical Consultants

PG&E Letter DCL-26-032

Response to DCISC Letter to Mr. Blair Jones, Dated May 5, 2026, Re: Recent
Meeting with the Chair of the California Energy Commission

PG&E Letter DCL-26-032

Dr. Per Peterson
c/o Diablo Canyon Independent Safety Committee
Office of Legal Counsel
SW 4th & Mission, Suite 2, P.O. Box 4523
Carmel, CA 93921-4523

Response to DCISC Letter to Mr. Blair Jones, Dated May 5, 2026, Re: Recent
Meeting with the Chair of the California Energy Commission

Dear Dr. Peterson:

Thank you for your letter regarding Diablo Canyon Independent Safety Committee (DCISC) discussions with the California Energy Commission regarding the status and plans for evaluation of Diablo Canyon Power Plant (DCPP) Unit 1 reactor surveillance Capsule B.

As background, Pacific Gas and Electric Company (PG&E) complies with Nuclear Regulatory Commission (NRC) regulations (10 CFR 50 Appendix H) that require a Reactor Vessel Material Surveillance Program for DCPP Units 1 and 2. The purpose of this program is to monitor vessel material changes over time through periodic withdrawal and testing of surveillance capsules installed in the reactor vessels to ensure safe operations. PG&E has satisfied the Reactor Vessel Material Surveillance Program requirements for DCPP Units 1 and 2 for the initial 40-year licensed operating period. As discussed in the DCPP License Renewal Application and independently confirmed by NRC in their safety evaluation report,¹ existing capsule data evaluations have demonstrated that both Unit 1 and 2 will meet the reactor vessel integrity criteria for 60 years. To support license renewal (operations for an additional 20 years), PG&E committed to withdraw and test one more Unit 1 surveillance capsule (Capsule B) to further confirm the integrity of the reactor vessel for at least 60 years of operations.

NRC regulation 10 CFR Part 50, Appendix H, requires "Each capsule withdrawal and the test results must be the subject of a summary technical report to be submitted...within eighteen months of the date of capsule withdrawal..."² As shown in the Enclosure, this corresponds to a summary technical report submittal requirement of October 2026. PG&E discussed in previous DCISC public meetings and fact findings (see the DCISC 35th Annual Report; the June 2025 public meeting slides and minutes³), that the Unit 1 Capsule B testing and analysis report

¹ See Sections 3.0.3.2.13 and 4.2 at <https://www.nrc.gov/docs/ML2515/ML25153A508.pdf>

² <https://www.ecfr.gov/current/title-10/chapter-I/part-50#Appendix-I-to-Part-50>

³ https://www.dcisc.org/event-public-meeting_47.htm

will be submitted to the NRC within 18 months, as required by 10 CFR Part 50, Appendix H, as described above. PG&E has been working to expedite the report; however, Westinghouse, the vendor performing the analyses, experienced delays due to equipment issues. PG&E will submit the report to DCISC concurrently with submission of the report to the NRC. Should the report be completed prior to the NRC 18-month due date, PG&E will submit the report earlier.

PG&E appreciates DCISC's engagement on this important topic and looks forward to discussing the results of the testing when they are available later this year.

Additionally, PG&E is offering to provide a confidential briefing of the analyses for DCISC members, and applicable consultants, prior to submittal of the owner accepted report to the NRC to assist them in preparation for future public meetings and independent review. The proposed confidential briefings are currently forecasted to occur in July 2026.

If you have any questions, please contact Mr. Tom Jones, Senior Director, at (805) 459-4530, or Mr. Philippe Soenen, Director, at (805) 459-3701.

Sincerely,



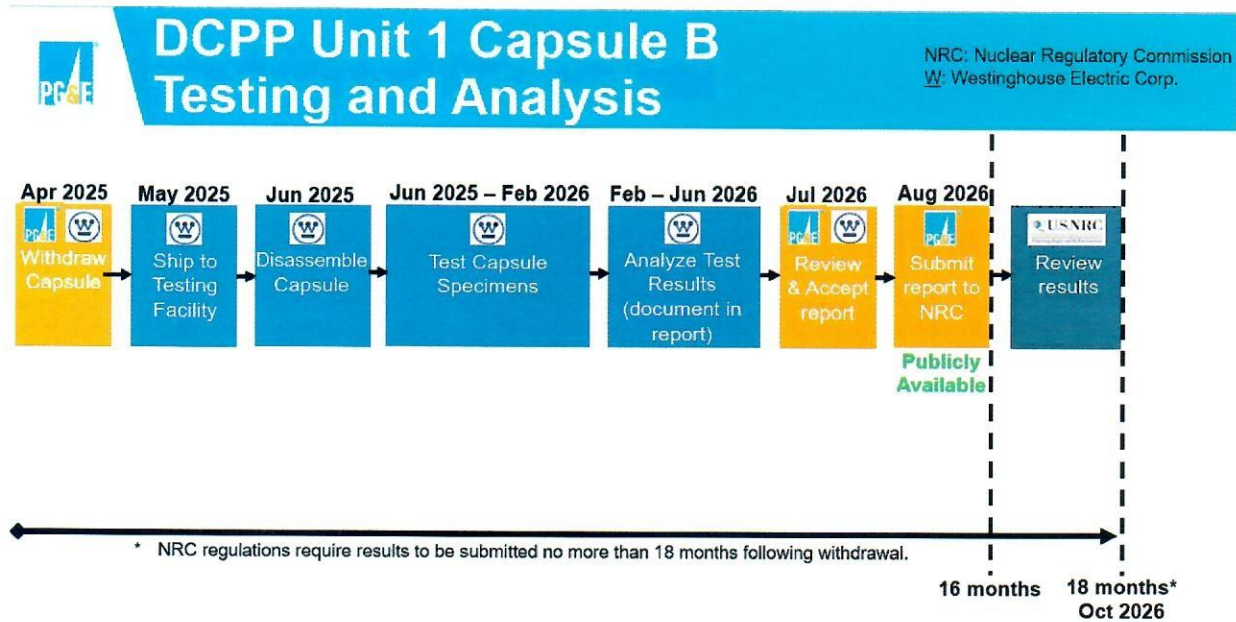
Blair C. Jones
Vice President, Business and Technical Services

May 15, 2026

Date

Enclosure

cc/Enc: Dr. Justin Cochran
Lisa Darden
David Hochschild
Dr. Andrew Kadak
Dr. Mark Kirk
Richard McWhorter, Jr.
Dr. Najmedin Meshkati
Robert Rathie
Dr. Raluca Scarlat
Richard St. Onge
R. Ferman Wardell
Robert Wellington





**Pacific Gas and
Electric Company**

Blair C. Jones
Vice President
Business and
Technical Services

Diablo Canyon Power Plant
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Avila Beach, CA 93424

805.545.3715
Blair.Jones@pge.com

PG&E Letter DCL-26-033

Mr. David Hochschild
California Energy Commission
715 P Street
Sacramento, CA 95814

Response to DCISC Letter to Mr. Blair Jones, Dated May 5, 2026, Re: Recent Meeting with the Chair of the California Energy Commission

Dear Mr. David Hochschild:

Pacific Gas and Electric Company (PG&E) recognizes your, and the public's, interest in the results of the Diablo Canyon Power Plant (DCPP) Unit 1 reactor surveillance Capsule B analysis results. Please find enclosed PG&E's response to the Diablo Canyon Independent Safety Committee (DCISC) Letter, dated May 5, 2026, Re: Recent Meeting with the Chair of the California Energy Commission.

In the response letter to DCISC, PG&E offered to provide a confidential briefing of the analyses for DCISC members, currently forecast to occur in July 2026. PG&E extends this invitation to you to participate in the proposed confidential brief with the California Energy Commission-appointed DCISC member or separately.

If you have any questions, please contact Mr. Tom Jones, Senior Director, at (805) 459-4530, or Mr. Philippe Soenen, Director, at (805) 459-3701.

Sincerely,

Blair C. Jones

May 15, 2026

Date

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